

# ***Development and Characterization of Ultra-High Performance Concrete for Dry Cask Applications***

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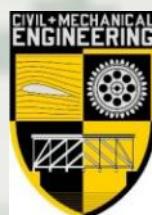
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# Outline

- Introduction
- Issues with dry cask storage
- Options for new materials selection
- Novel ultra-high performance concrete materials
- Materials development
- Microstructural characterization



# Nuclear Reactor Core



# Wet Pool Storage



## Independent Spent Fuel Storage Installation (ISFSI)



# An Application

- 2,000 MT of spent nuclear fuel (SNF) produced per year
- Stockpile as of 2011: 65,000 MT of SNF primarily being stored on-site in spent-fuel pools and dry casks
- Findings of a National Research Council study: Safety and Security of Commercial Spent Nuclear Fuel Storage, 2006.
  1. “Less spent fuel is at risk in an accident or attack on a dry storage cask than on a spent fuel pool.”
  2. “The potential consequences of an accident or terrorist attack on a dry cask storage facility are lower than for a spent fuel pool.”
  3. “The recovery from an attack on a dry cask would be much easier than the recovery from an attack on a spent fuel pool.”



# Dry Cask Design Considerations/NRC Licensing Requirements

- Structural Resilience- SNF must be protected in Earthquakes, tornados, floods, tsunami, lightning, fire, and explosion loading.
- Radiation Attenuation- Minimize exposure to people working with spent nuclear fuel. 5rem at the fence line.
- Heat Removal Capability- sufficient for normal and accident conditions
- Criticality- Neutron absorber must last as long as cask.
- Activation- “Provisions must be made to...minimize the quantity of radioactive wastes and contaminated equipment” (10 CFR 72.130 Criteria for Decommissioning)

Current dry-cask storage system designs are licensed for 20 years, with a reasonable expectation for relicensing for an addition 20 years, but after that...



# Literature of Materials for Shielding

- Work on optimizing concrete for shielding has been ongoing since at least the 1950s with heavy-weight aggregates and materials with boron a primary focus

**TABLE 5—PROPERTIES OF VARIOUS CONCRETE MIXTURES FOR RADIATION SHIELDING\***

Type of concrete	Percentage of concrete, by weight				Unit weight		Half-thickness, cm		Diffusion length, cm	
	Water†	Cement	Fine aggregate	Coarse aggregate	g per cu cm	lb per cu ft	Neutrons	γ-rays		
Ordinary	8.4	17.7	27.6	46.3	—	2.35	146.7	10.7	13.3	5.6
Ordinary + 1.25 percent Pyrex	6.05	18.37	27.78	46.55	—	2.39	149.2	7.4	13.3	3.3
Magnetite aggregate	5.5	11		83.5	51.06	3.78	236.0	8.5	9.0	2.9
Limonite aggregate	10.5	21		68.5	35.36	2.63	164.2	4.8	9.0	2.5
Limonite + scrap iron	4.71	9.54	26.35	59.4	73.24	4.41	275.3	5.4	7.9	2.5
Limonite + scrap iron + 0.7 percent Pyrex	12.8	12.3	32.8	41.4	62.78	3.60	224.7	5.0	7.7	2.3

\*Data from reference 8.

†Water added only; water present in limonite ore not counted.



From: Callan, Edwin J., Concrete for Radiation Shielding, Journal of the American Concrete Institute, Title No. 50-2, Sept., 1953

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# Optimum concrete for radiation shielding...

1. Minimized volume and weight for required level of attenuation
2. Relatively easy to construct with very high probability of homogeneity
3. Relatively long life expectancy
4. Low activation – easily recycled
5. Cost effective

## Optimum concrete for ballistic penetration resistance... very similar objectives

### Relevant current concrete technologies...

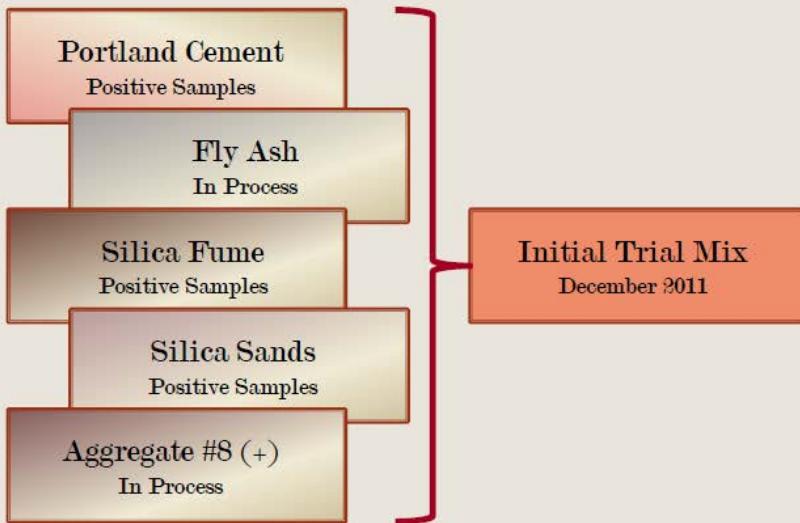
1. Self-Consolidating Concrete (SCC)
2. Fiber-Reinforced Concrete (FRC)
3. Ultra-High-Performance Concrete (UHPC)
4. High-Strength-High-Ductility Concrete (HSHDC)\*

\*From: Ravi Ranade, Victor C. Li, Michael D. Stults, William F. Heard and Todd S. Rushing, "Composite Properties of High-Strength, High-Ductility Concrete," ACI Materials Journal, 110-M37, July-August, 2013



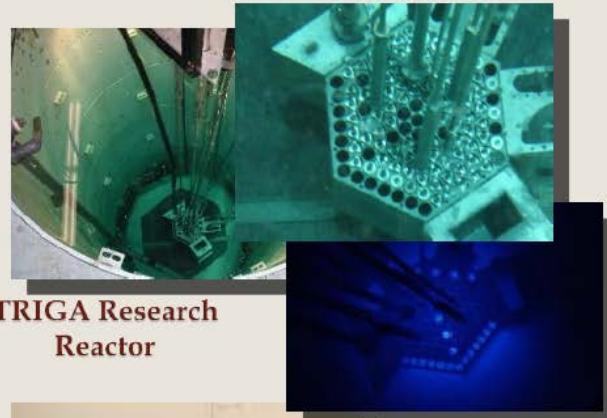
# More recent work...

## Our Initial Research Results



Long term testing to simulate Nuclear Reactor Lifespan Planned To Begin Spring 2012

### University of Utah Nuclear Facility



TRIGA Research Reactor



Counting Laboratory

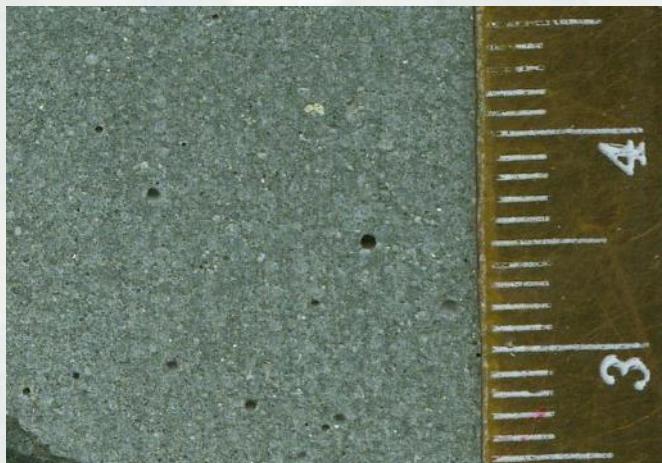
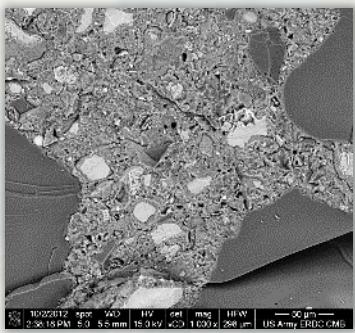
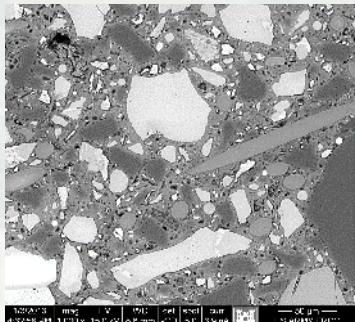
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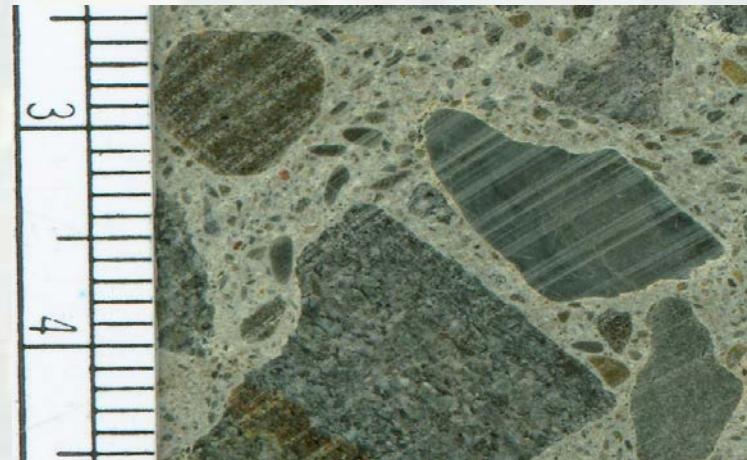
From: Rapich et al, Radiation Resistant Concrete, ACI Fall Convention, 2011

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# UHPC vs. NSC



Cross Section of UHPC



Cross Section of NSC

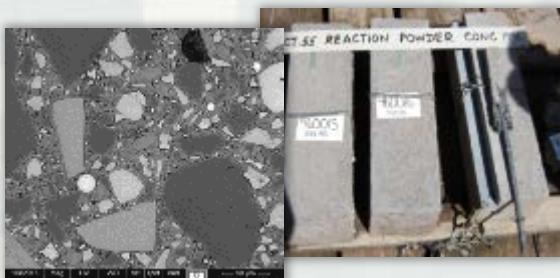
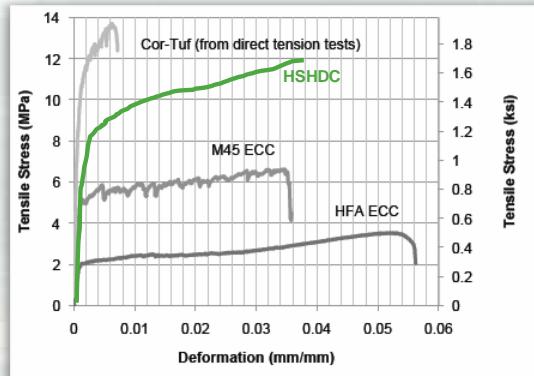
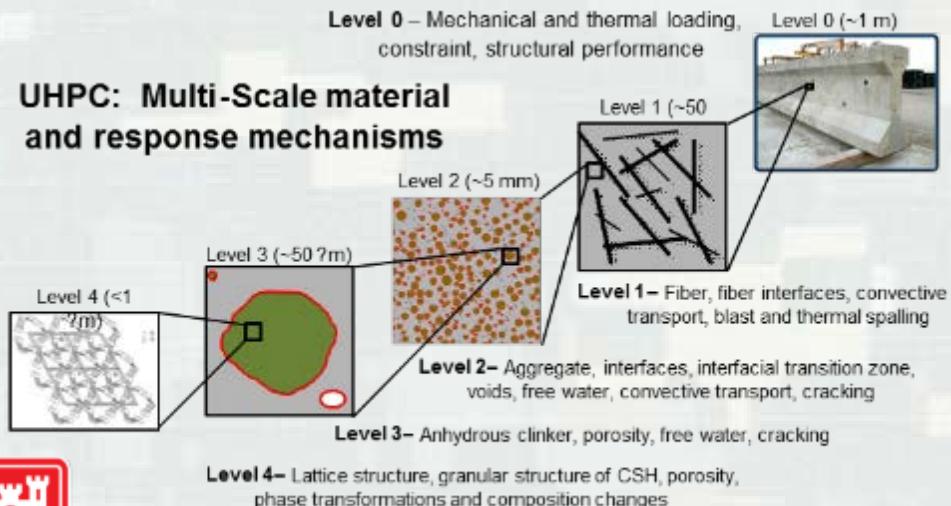
Concrete Properties			
	UHPC	High Strength	Normal Strength
Compressive Strength	> 150 MPa (21.7 ksi)	>41.4 Mpa (>6 ksi)	<41.4 MPa (<6 ksi)
Density	2500 kg/m <sup>3</sup> (156 lb/ft <sup>3</sup> )	Slightly higher than NSC	2300 kg/m <sup>3</sup> (144 lb/ft <sup>3</sup> )



# Ultra-High Performance Concretes (UHPC)

Decades of R&D on development, characterization, testing, modeling, and fielding of UHPCs:

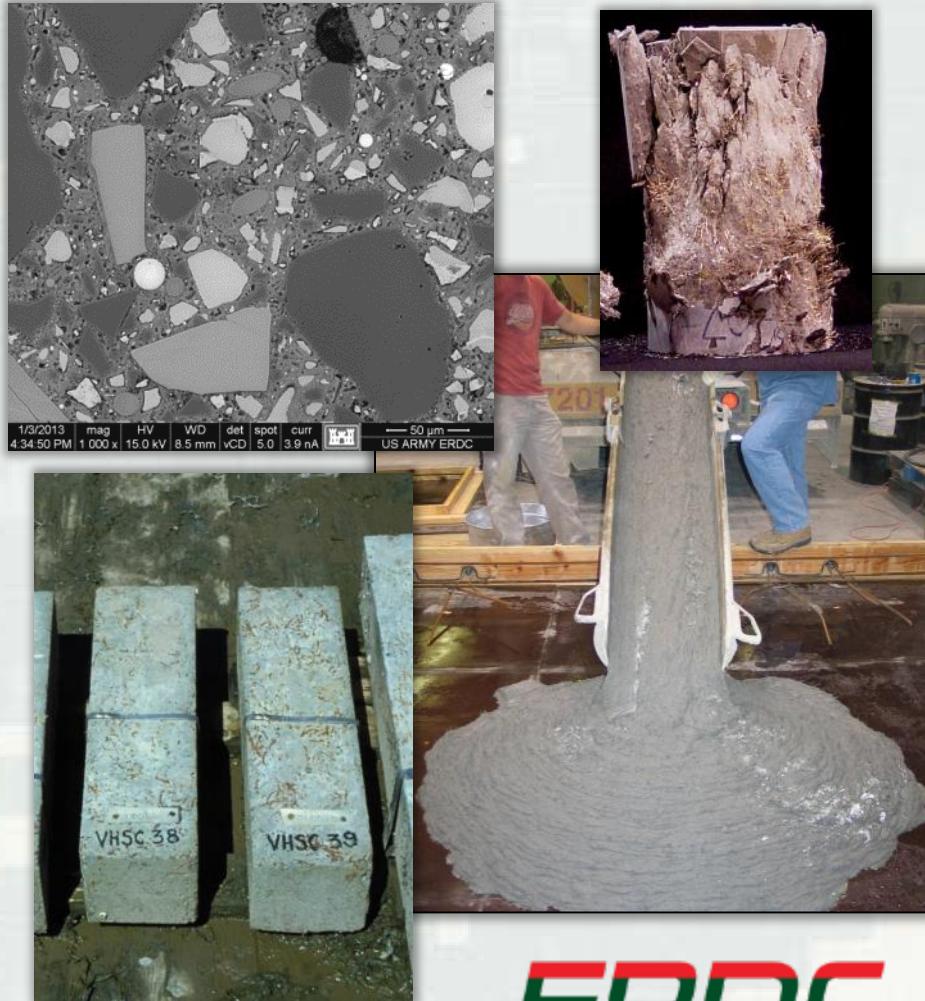
- The Corps' Cor-Tuf UHPC formulation
- Strengths in excess of 200MPa / 30ksi
- High toughness and durability
- High-strength, high-ductility formulations



# History WES/ERDC UHPC Research

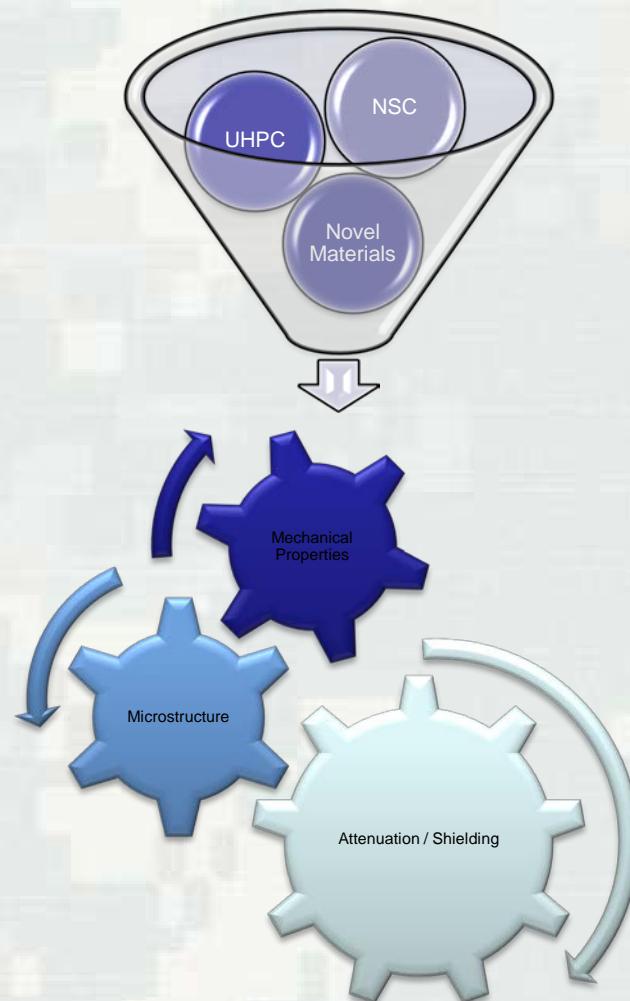
## Very-High-Strength Concrete - VHSC

- “Local” materials used
- Not RPC, incorporating ASTM C 33 concrete sand
- Developed by Donald M. Walley and Billy Neeley
- Early/Mid 1990's
- **Mixture #247!**
- 200 MPa (29,000 psi) with steam curing at 190° F
- **Revolving-drum mixer truck used to produce!**
- **Lead to Cor-Tuf ® UHPC**

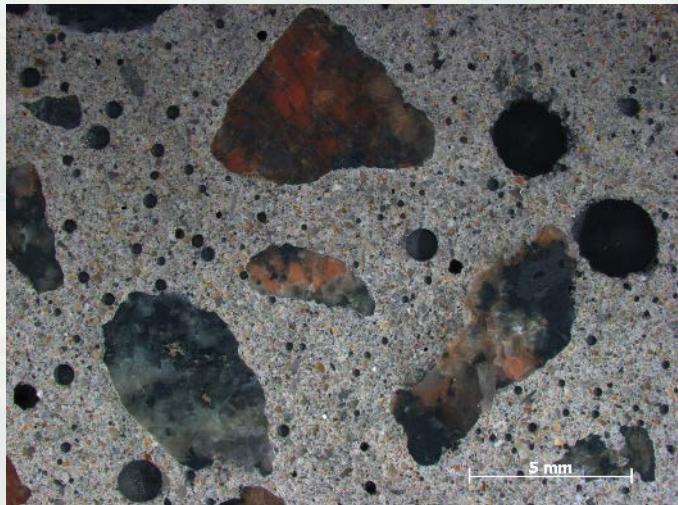


# New Materials Development and Testing

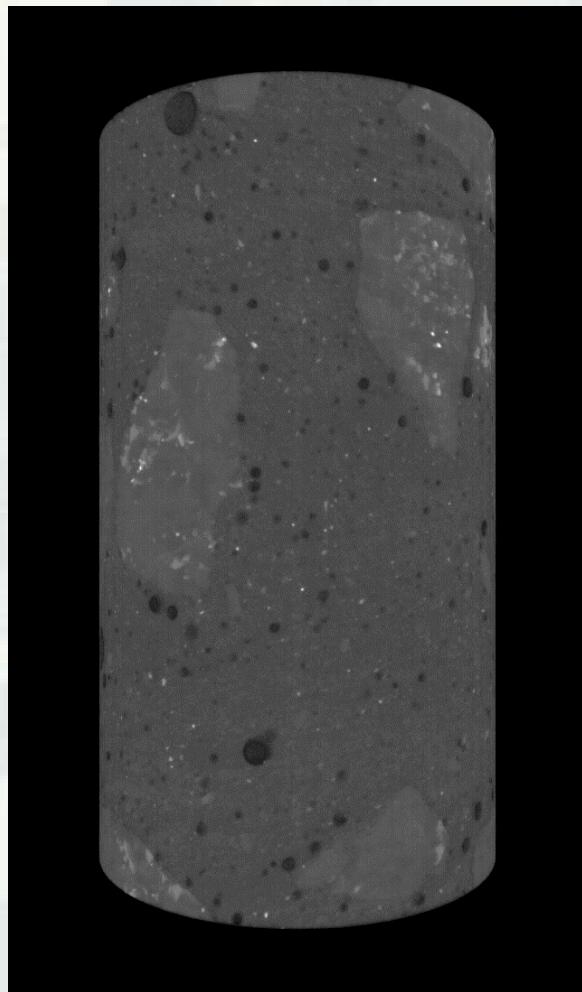
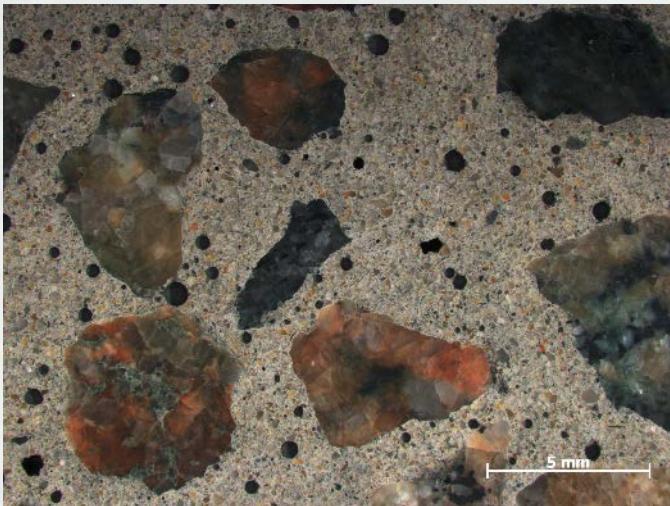
- Adapt existing Cor-Tuf UHPC formulation
- Increase density using steel “sand”
  - Material obtained from powder metal supplier
- Replace silica sand with steel “sand”
- Include steel fibers with steel “sand”
- NSC control material



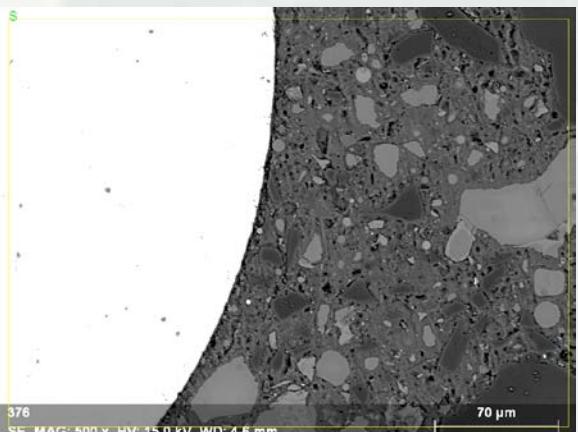
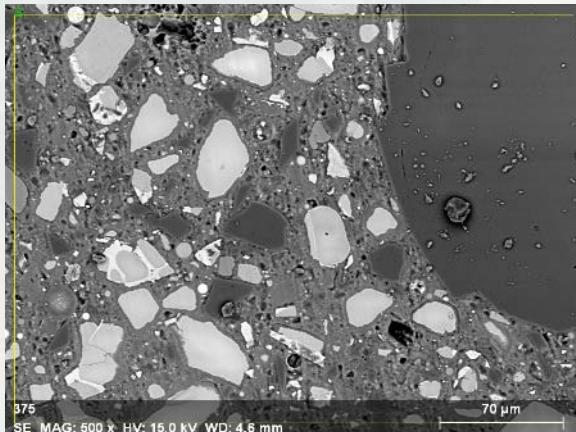
# Microstructural Characterization of NSC



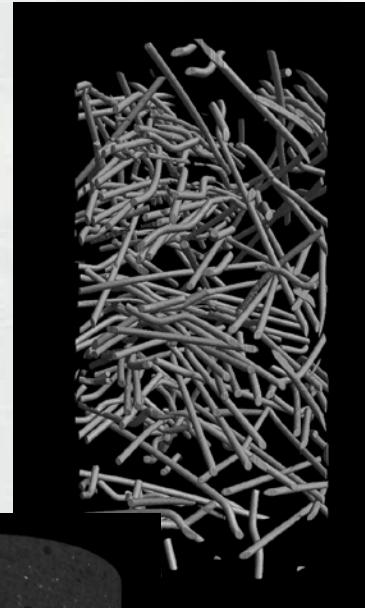
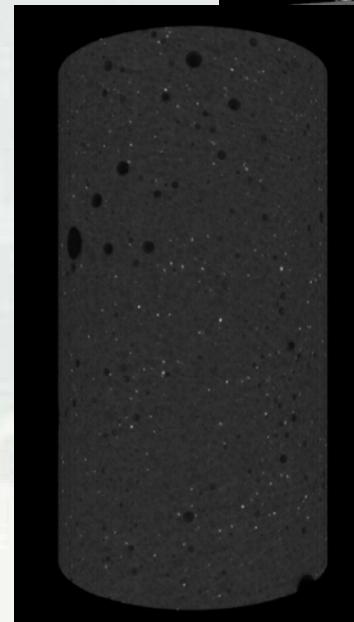
Typical  
microstructure,  
coarse, fine agg,  
paste, and  
porosity



# Microstructural Characterization of Cor-Tuf UHPC



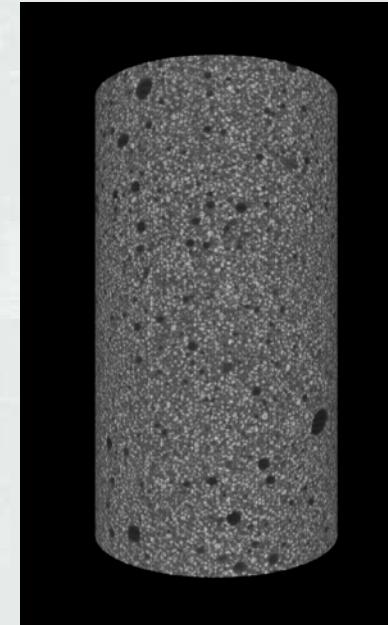
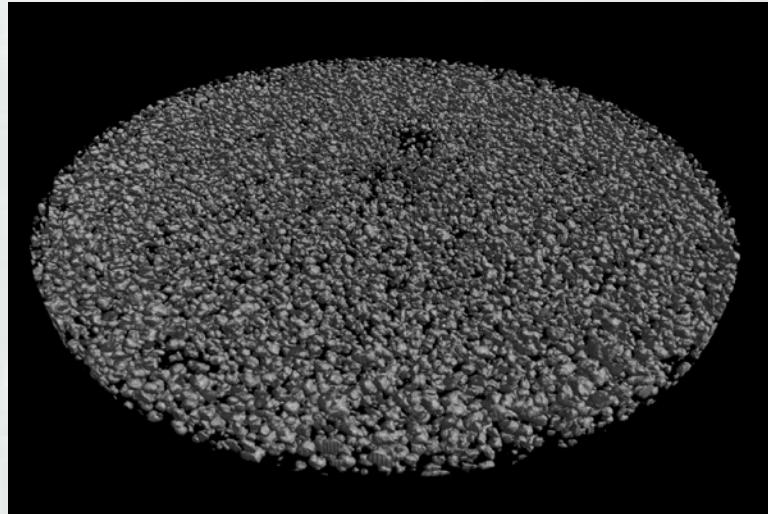
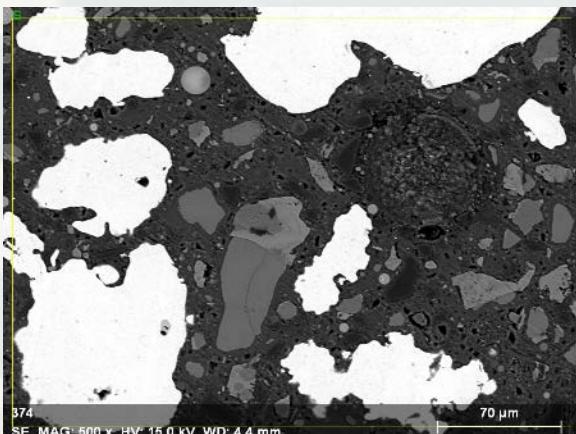
Typical UHPC  
microstructure, fine  
aggregate, silica flour,  
paste, fibers



# Microstructural Characterization of Cor-Tuf UHPC with Steel “Sand”



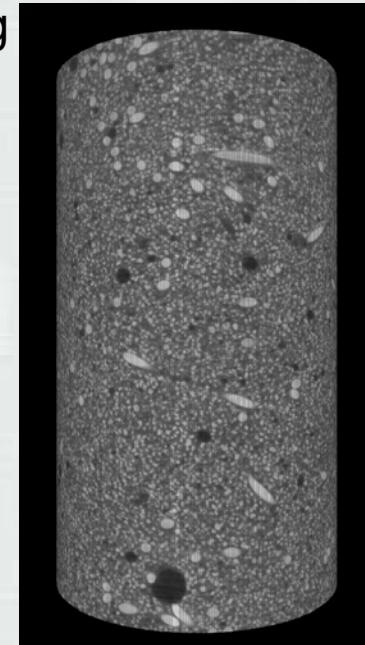
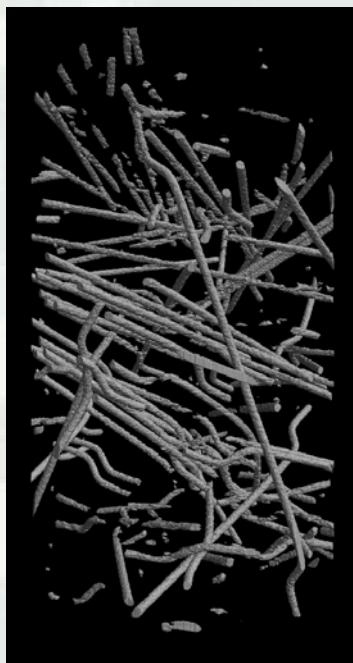
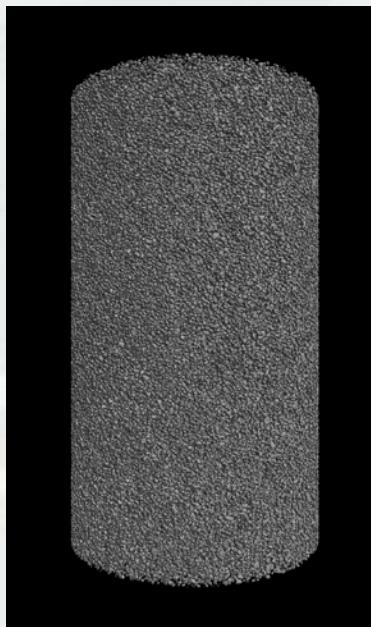
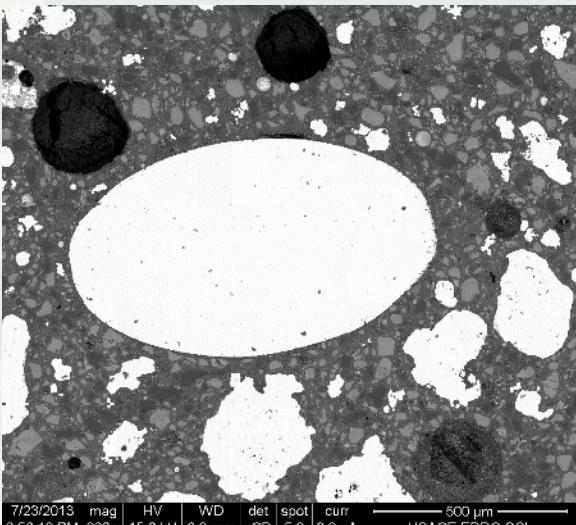
Typical UHPC  
microstructure without  
fibers but including steel  
“sand” particles



# Microstructural Characterization of Cor-Tuf UHPC with Steel “Sand” and Fibers



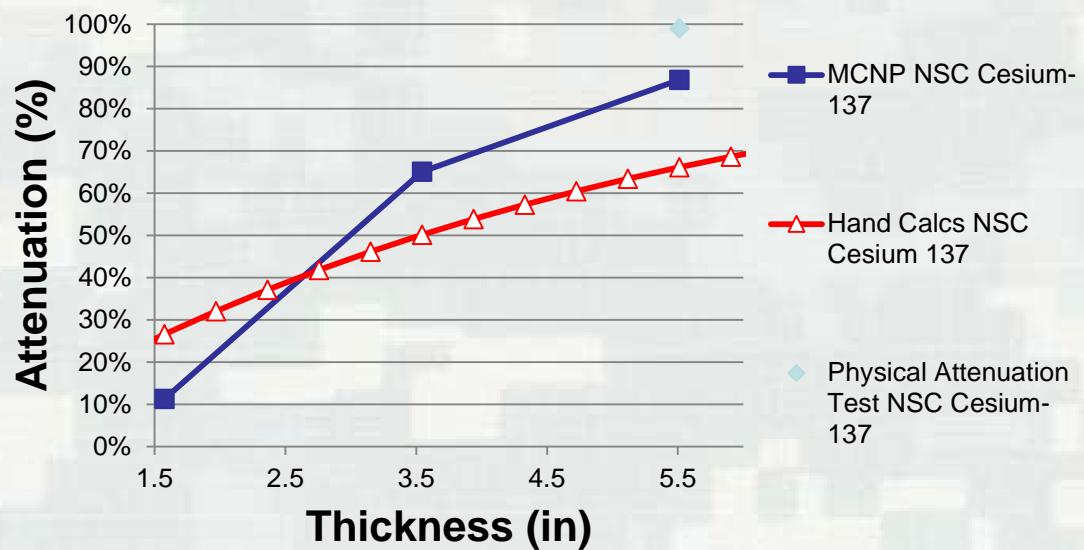
Typical UHPC microstructure including steel fibers and steel “sand”



# UHPC in Dry Storage of Spent Nuclear Fuel

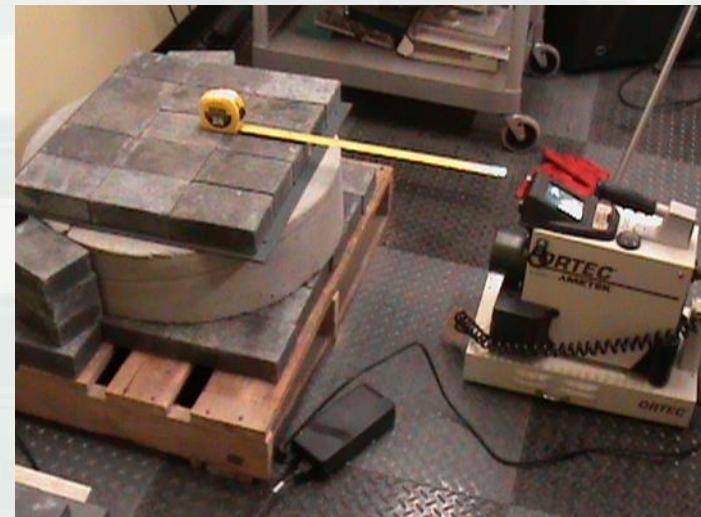
Attenuation Modeling with  
Monte Carlo N Particle (MCNP)  
Transport Code

## Attenuation vs. Concrete Thickness

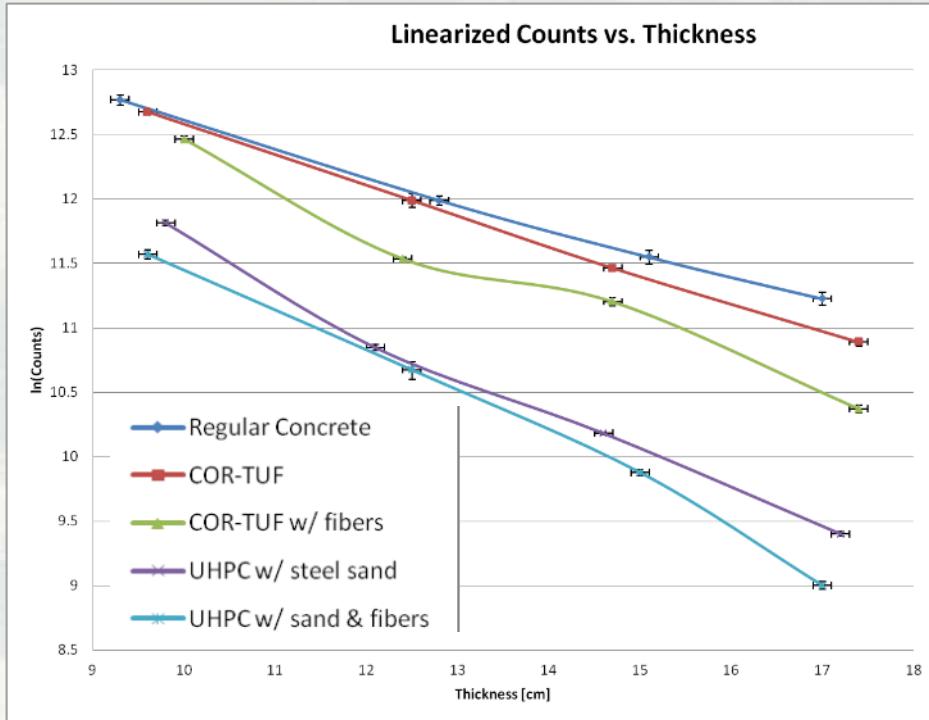


Chemical composition of UHPC required as input for simulation.

Porosity distribution also characterized.



# Material Optimization of the Dry Cask Storage System: Comparison of Attenuation Coefficients

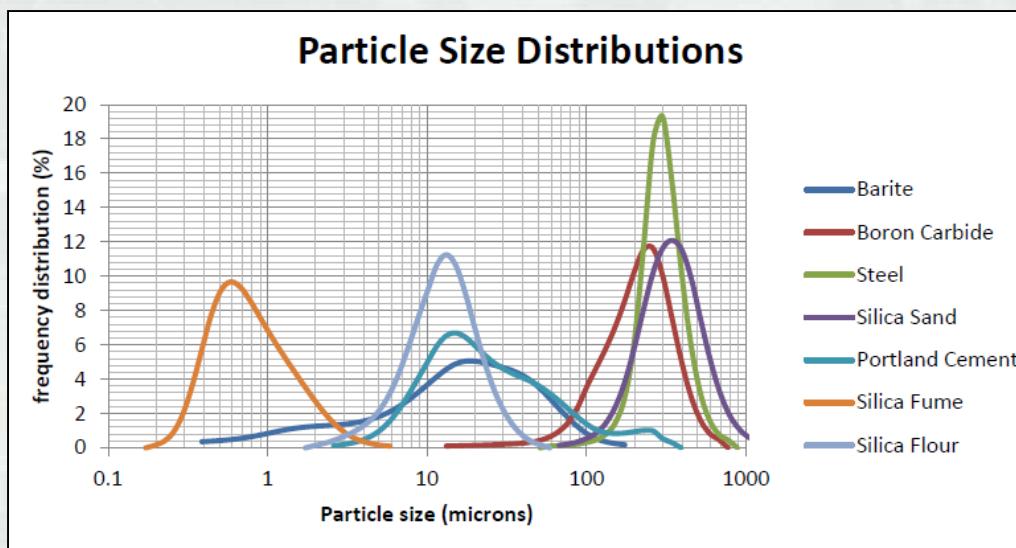


## A Study of Concrete Additives to Enhance Neutron Attenuation for Dry Cask Storage



## Optimization of Ultra High Performance Concrete(UHPC) for Multi-Scale Penetration Resistance

UHPC Additive	Effective Macroscopic Cross Section [cm <sup>-1</sup> ]	Uncertainty
Glass Microspheres	0.05	0.01
Boron Carbide	0.062	0.009
Barite	0.079	0.008
Steel Sand	0.061	0.009
Silica Sand	0.055	0.005



	Unit Weight pcf	f'c ksi MPa
Silica	140	29 201
Glass Bubbles	110	14 96
Steel	210	37 254
Boron Carbide	135	27 186
Barite	155	32 219



# Summary

- UHPCs are a unique class of materials with dense microstructure, low interconnected porosity, high strength, and high toughness
- Adapting UHPCs for shielding applications presents many unique properties, particularly with steel “sand”
- Improvements in attenuation of up to 50% observed in various testing along with improvements in mechanical properties (strength)
- Other dense particles can also be incorporated into UHPC to improve properties



# Future Work

- Continue/expand the literature review.
- Resolve issues with the MCNP modeling.
- Utilize MCNP to guide optimization of mix design.



# Thank you! Questions?



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