Microstructural Tailoring of Ferritic/Martensitic Grade 91 Steel Using Wire Arc Additive Manufacturing

T.M. Kelsy Green¹; Niyanth Sridharan^{2a}; Xiang (Frank) Chen²; Kevin Field^{2b}

¹University of Michigan-Ann Arbor

²Oak Ridge National Laboratory

aCurrently at Lincoln Electric, India

^bCurrently at University of Michigan-Ann Arbor





Extreme Nuclear Environments

Advanced Reactor Design Considerations for Structural Materials Advanced Reactor Radiation Effects

Desired Material Features

Fine, stable precipitates

Temp. Range: 300-1,000°C

Damage Level: >200 dpa

Stress: ~150 MPa

Corrosion

 Radiation hardening and embrittlement

- Phase instabilities
- Irradiation creep
- Swelling
- He embrittlement

Good longterm thermal aging

radiation embrittlement

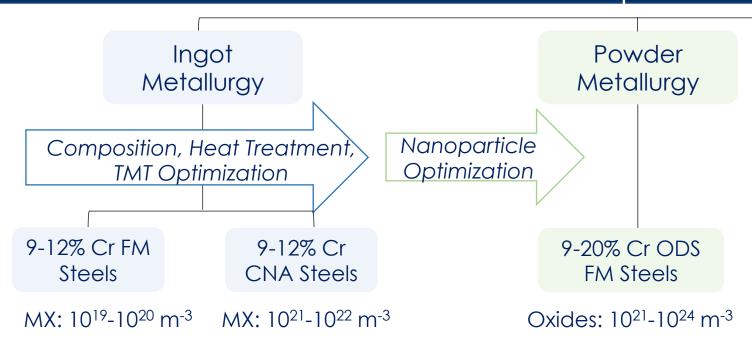
Resist

Creep resistance

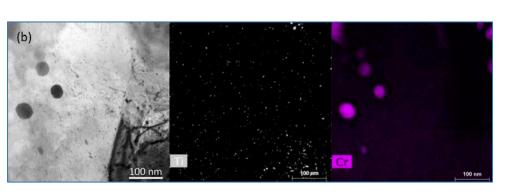
Resist void swelling

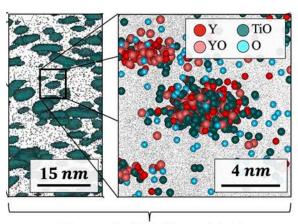
High strength & good ductility

9-20% Cr Alloy Development for Nuclear Reactor Core Internal and External Components



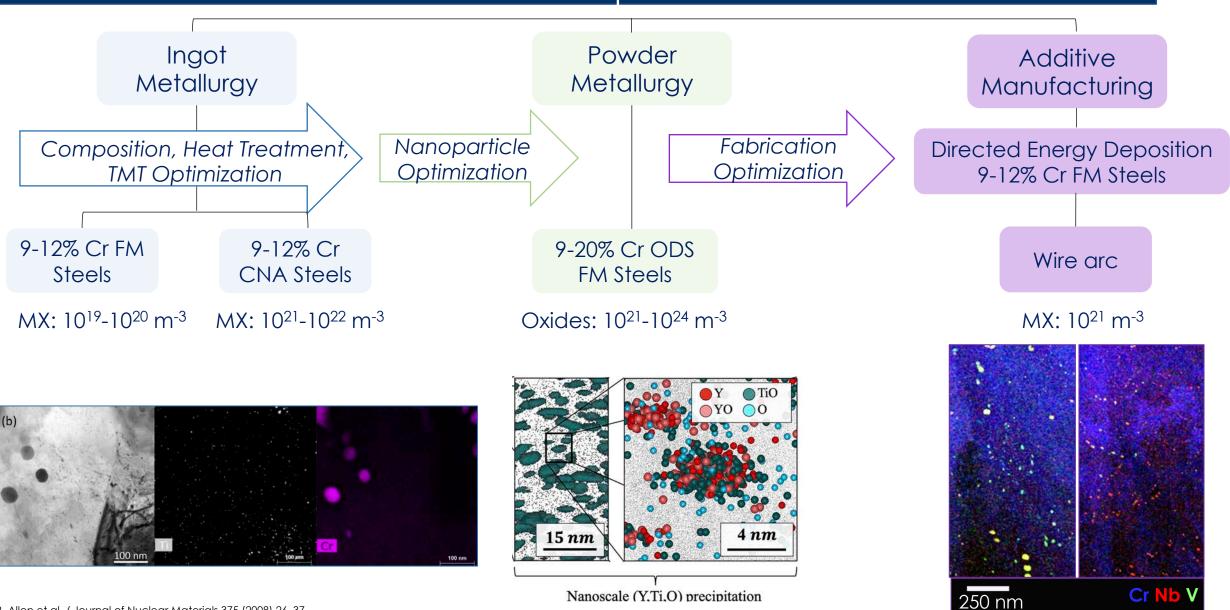
How can we efficiently fabricate FM steels with high nanoparticle densities?





Nanoscale (Y.Ti.O) precipitation

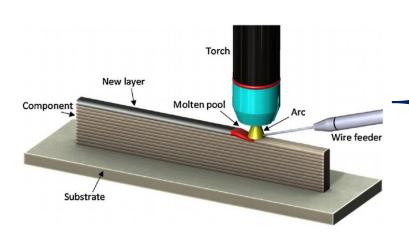
9-20% Cr Alloy Development for Nuclear Reactor Core Internal and **External Components**



T.R. Allen et al. / Journal of Nuclear Materials 375 (2008) 26–37 L. Tan et al. / Journal of Nuclear Materials 509 (2018) 267-275 C.P. Massey et al. / Journal of Nuclear Materials 522 (2019) 111-122

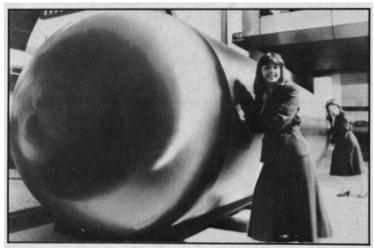
WAAM is preferred technique for RPV Fabrication

Wire arc AM process



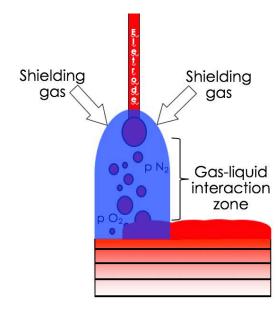
High deposition rates achievable

Fabricate large, noncomplex geometries



58 metric ton pressure vessel with dome fabricated with submerged arc welding (SAW)

Tailor structure of materials via control of processing parameters



This experiment aims to:

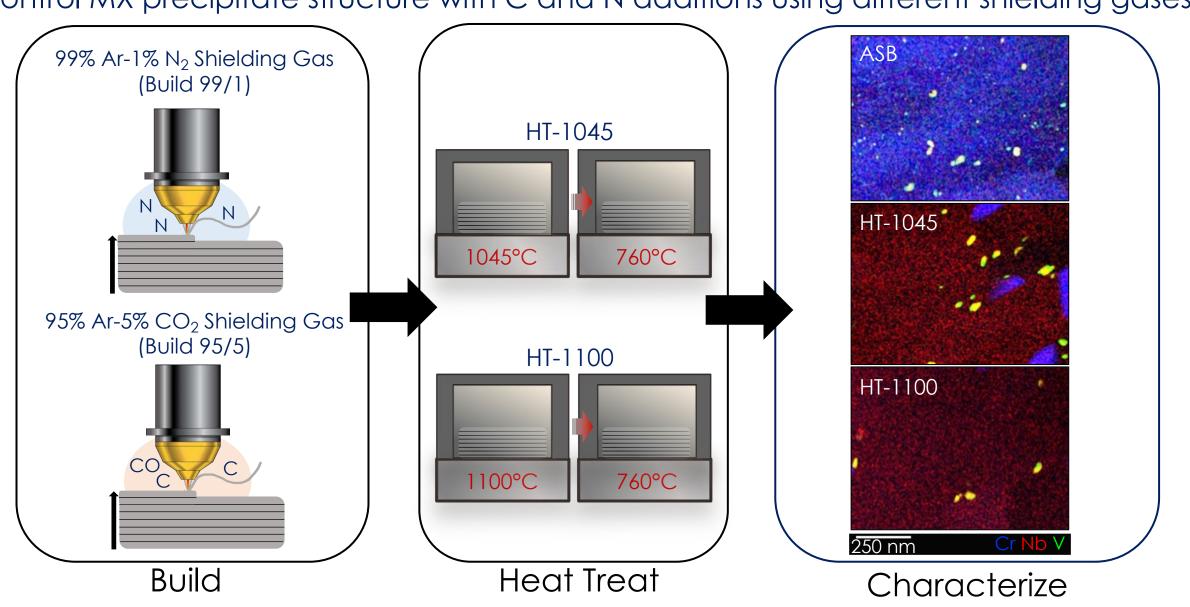
Assess WAAM for FM fabrication

Mimic or improve upon wrought properties

Control precipitate structure with AM processing parameters

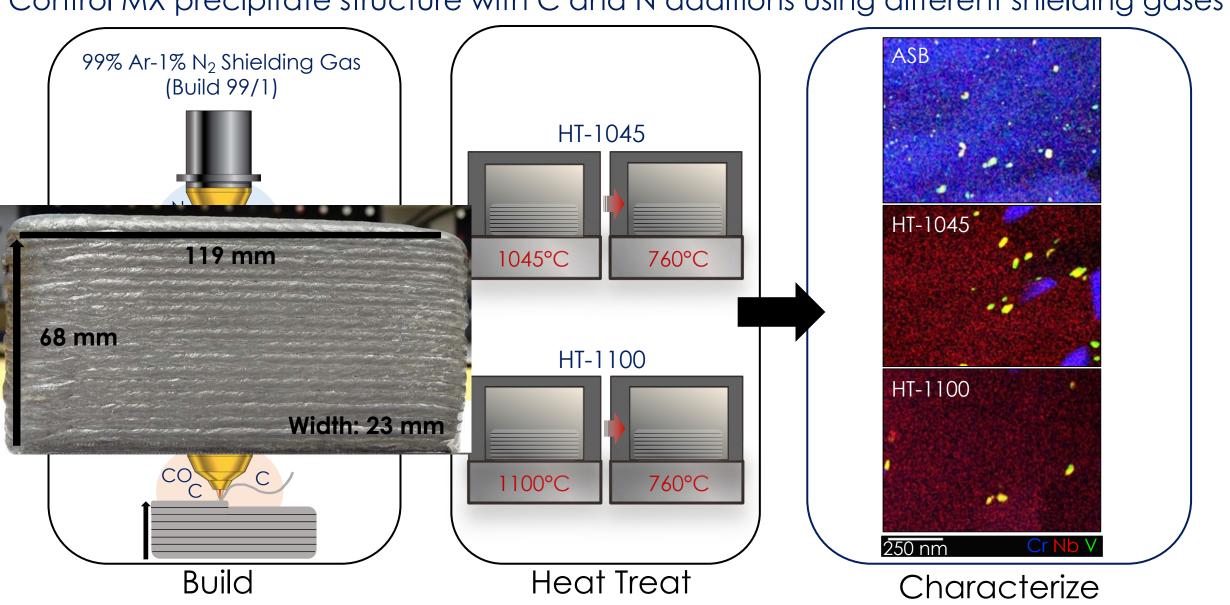
WAAM Grade 91 Experimental Details

Control MX precipitate structure with C and N additions using different shielding gases



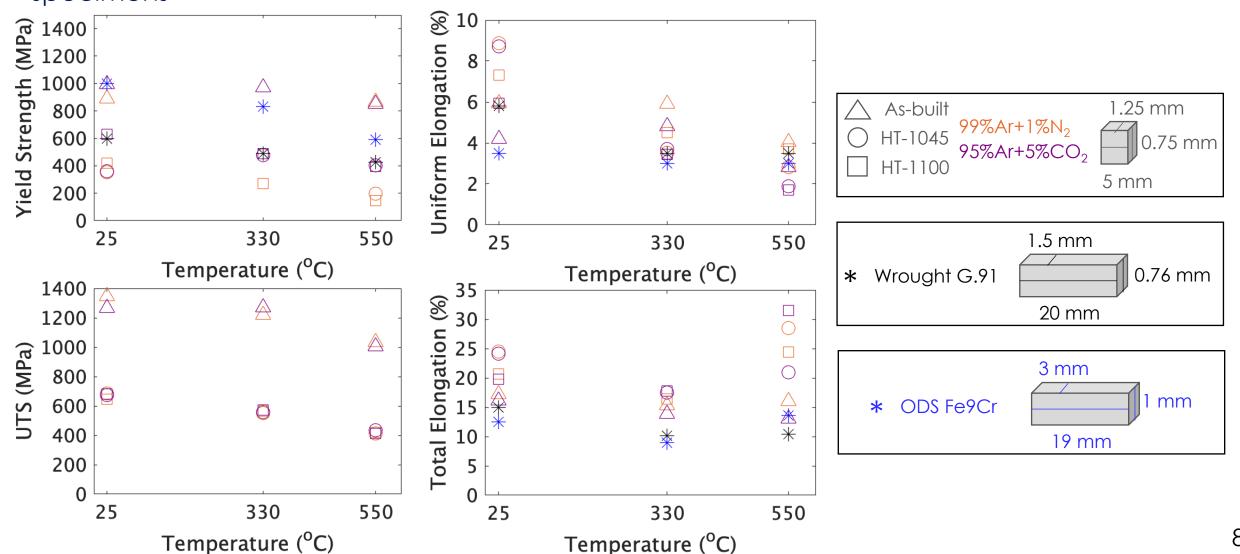
Wire Arc AM Experimental Details

Control MX precipitate structure with C and N additions using different shielding gases



Mechanical Properties

- Shielding gases caused no difference in strength
- All as-built specimens had higher strength than the heat-treated AM, wrought, and ODS Fe9Cr specimens



General Microstructure of As-Built G. 91

Representative of wrought FM steels: laths, packets, and prior austenite grain boundaries

Build 99/1 Build 95/5 99/1 ASB 99/1 ASB 95/5 ASB 95/5 ASB Prior y grain boundary As-Built $100 \mu \text{ m}$ 95/5 HT-1100 95/5 HT-1100 /1 HT-1100 99/1 HT-1100 Heatreated 500 nm

Shielding Gas Effect on Composition

Element	Initial Content (wt. %)	Build 95/5 Content (wt. %)	Build 99/1 Content (wt. %)	
0	0.008	0.0316	0.0178	
С	0.08	0.093	0.072	
N	0.04	0.0386	0.0648	

Build 95/5

16.25% increase in C

Build 99/1

62% increase in N

How do these changes compare to values from literature?

ASME Code for Wrought Grade 91

C

0.06-0.15 wt.%

N

0.025-0.08 wt.%

Welded Grade 91

SAW: no shielding gas

18% decrease in C

34% decrease in N

C content Base Material	C Content from welding	
<0.1wt.%	increase	
>0.1wt.%	decrease	

GTAW: 95% Ar+5% CO₂

38% increase in C

GTAW: 100% Ar

63% increase in C

GTAW: 100% CO₂

38% increase in C

GTAW: 99.7% Ar

27% decrease in C

Welded Fe

Auto-GTA: 95% Ar+5% N₂

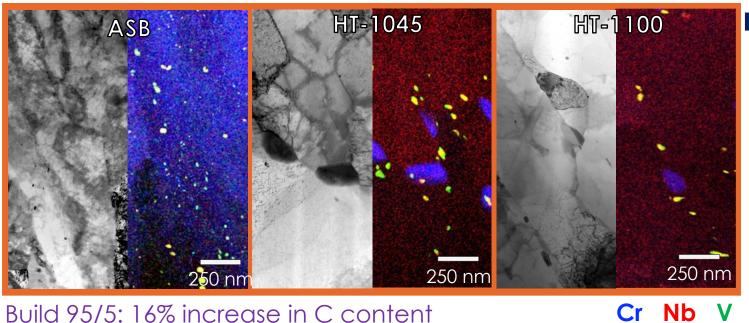
1,800% increase in N

Auto-GTA: 80% Ar+20% N₂

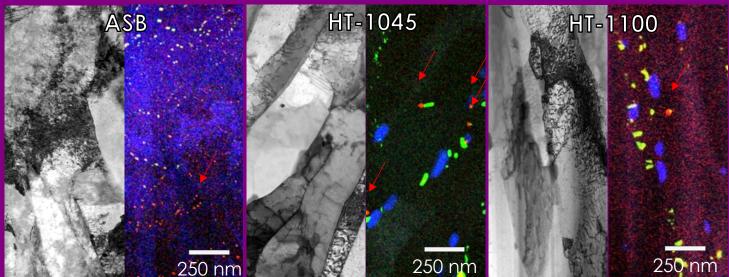
2,900% increase in N

Shielding Gas Effect on MX Precipitation

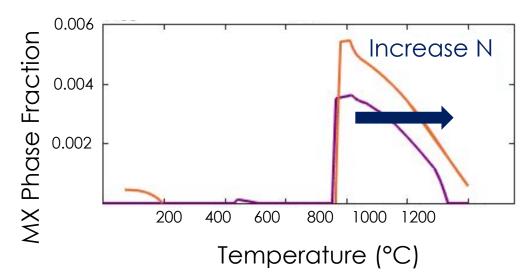
Build 99/1: 62% increase in N content







Controlled MX precipitate chemistry and temperature stability with shielding gas composition



Shielding Gas Used MX Stability Range

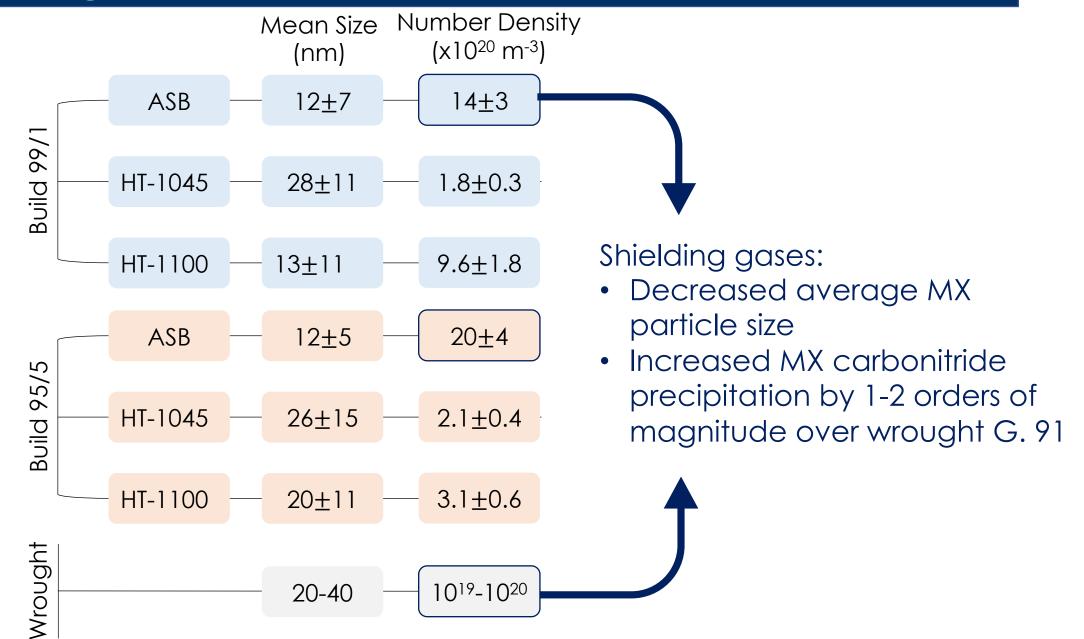
99%Ar+1%N₂

765->1,200°C

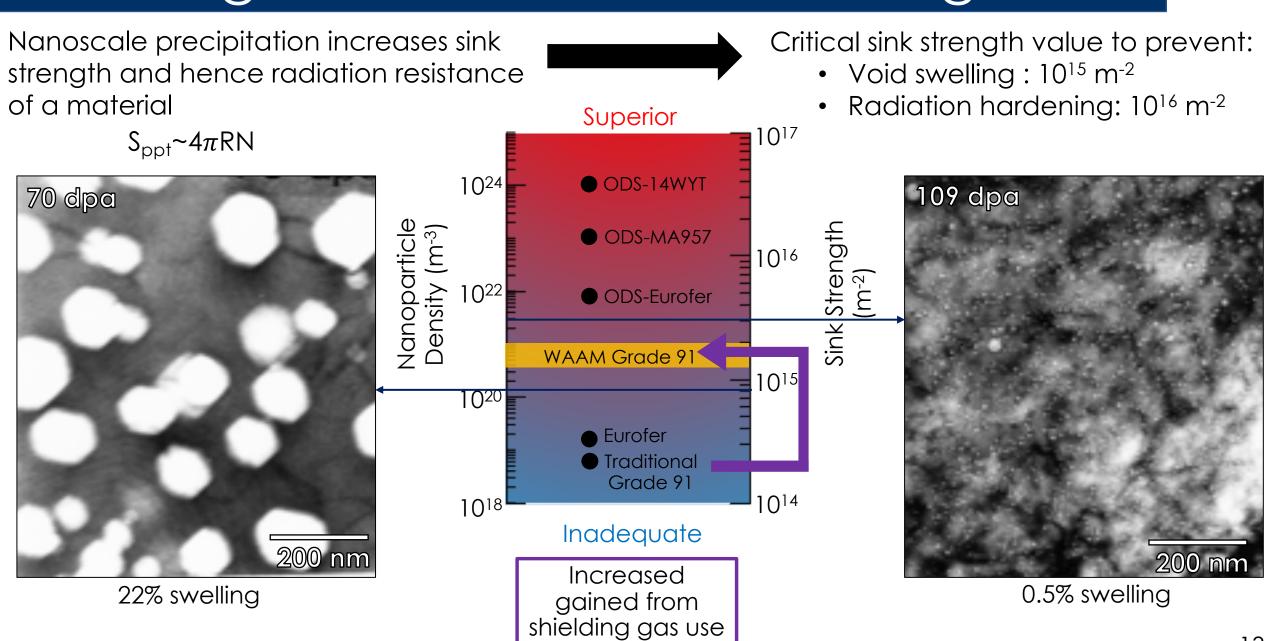
95%Ar+5%CO₂

755-1,130°C

Shielding Gas Effect on MX Precipitation



Shielding Gas Effect on Sink Strength



Summary

Conclusion

Successfully fabricated FM steel Grade 91 with DED-AM technique

Improved upon wrought Grade 91 microstructure and mechanical properties

Small tweaks in the processing parameters during AM can produce vastly different microstructures, allowing for creative opportunities for hybrid materials processing

Proved ability to control MX precipitate chemistry and stability with shielding gas composition

Increased sink strength of Grade 91 using shielding gases that supersaturated C and N in the microstructure during AM

T.M. Kelsy Green:

tmkgreen@umich.edu

References

- 1. K. Sawada et al., "Microstructure characterization of heat affected zone after welding in Mod.9Cr–1Mo steel," Materials Characterization, Vol. 101, (2015) pp. 106-113
- 2. C. Pandey et al., "Characterization of Microstructure of HAZs in As-Welded and Service Condition of P91 Pipe Weldments," Met. Mater. Int., Vol. 23, No. 1 (2017), pp. 148-162
- 3. C. Pandey et al., "Homogenization of P91 weldments using varying normalizing and tempering treatment," Materials Science & Engineering A 710 (2018) 86–101
- 4. B. Arivazhagan, "A study on influence of shielding gas composition on toughness of flux-cored arc weld of modified 9Cr-1Mo (P91) steel," Journal of Materials Processing Technology 209 (2009) 5245–5253
- 5. C. Pandey et al., "Autogenous Tungsten Inert Gas and Gas Tungsten Arc With Filler Welding of Dissimilar P91 and P92 Steels," J. of Pressure Vessel Technology 140 (2018) 021407-1-021407-7
- 6. B. Arivazhagan, "A comparative study on the effect of GTAW processes on the microstructure and mechanical properties of P91 steel weld joints," Journal of Manufacturing Processes 16 (2014) 305–311
- 7. Z. Zhange et al., "Recent developments in welding consumables for P(T)-91 creep resisting steels," International Conference on Integrity of High Temperature Welds, 3-4 November 1998, Nottingham, UK
- 8. R.L. Klueh & D.R. Harris, High-chromium Ferritic And Martensitic Steels For Nuclear Applications, ASTM Stock Number: MONO3, 2001.
- 9. R. L. Klueh & D. J. Alexander, "Heat treatment effects on impact toughness of 9Cr±1MoVNb and 12Cr±1MoVW steels irradiated to 100 dpa," Journal of Nuclear Materials 258-263 (1998) 1269-1274