聚变能源概论

高喆

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第8讲:

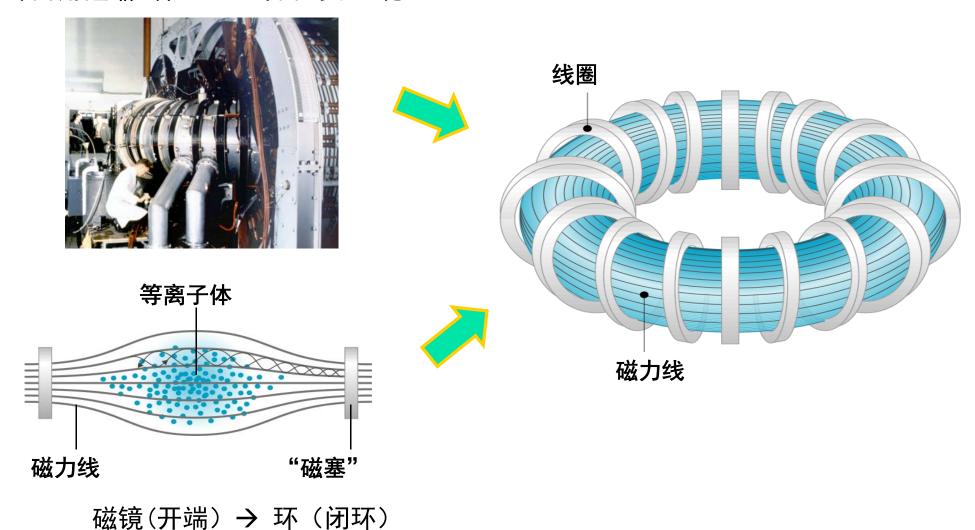
受控聚变的基本途径1-磁约束(续)

上节回顾

- 约束的图像和方式
- 磁约束聚变基本原理
- 箍缩装置: 瞬态快过程
- 磁镜/最小场: 开端损失

环形约束: 殊途同归的选择

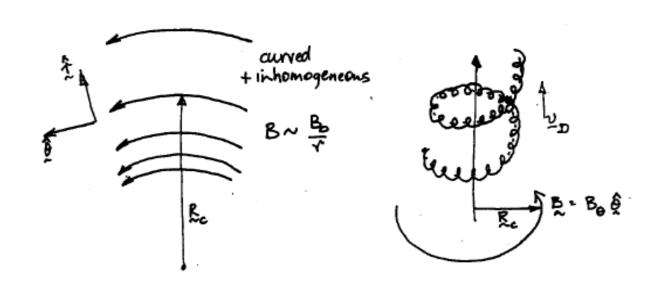
环向角箍缩(瞬态)→环向约束(稳态)



$$\mathbf{v}_{\nabla B} = \frac{(-\mu_B \nabla B) \times \mathbf{B}}{qB^2} \qquad \mathbf{v}_R = \frac{\left(-mv_{//}^2 \mathbf{R}\right) \times \mathbf{B}}{qR^2B^2} = \frac{mv_{//}^2 \mathbf{B}}{qB} \times \left(\frac{\mathbf{B}}{B} \cdot \nabla\right) \frac{\mathbf{B}}{B}$$
$$B\nabla B = (\mathbf{B} \cdot \nabla) \mathbf{B} - (\nabla \times \mathbf{B}) \times \mathbf{B}$$

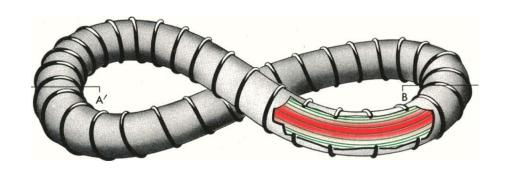
环漂移

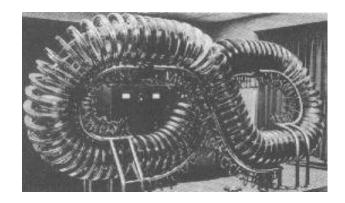
$$v_T = v_{\nabla B} + v_R = \frac{m}{q} \left(\frac{R_c \times B}{R_c^2 B^2} \right) \left(v_{\parallel}^2 + \frac{1}{2} v_{\perp}^2 \right).$$



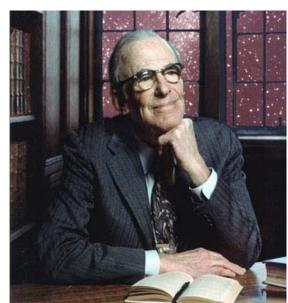
no confinement!

仿星器

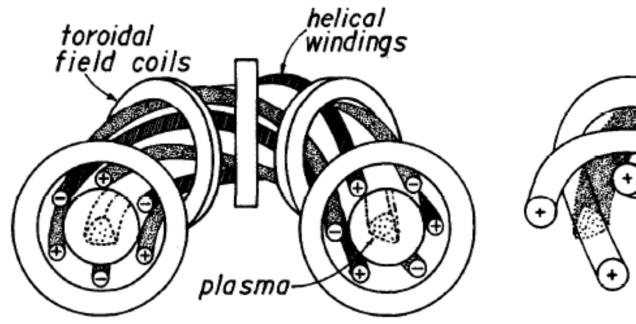




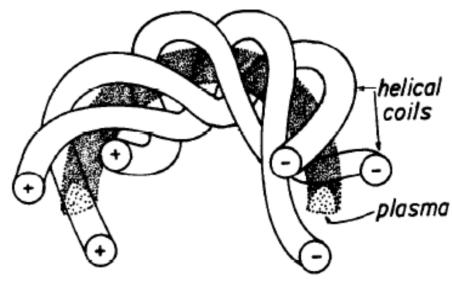
Model B-3 stellarator @US



Lyman Spitzer, Jr. (1914—1997)



Original stellarators

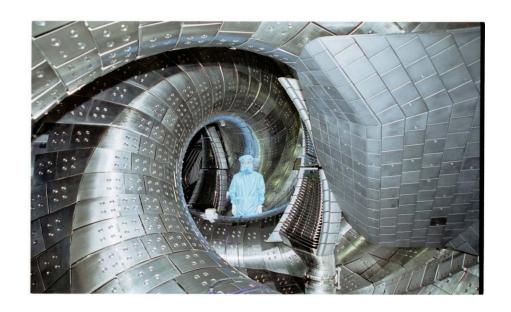


Torsatron or Heliotron

Large Helical Device (LHD) in Toki, Japan

 $T_i(0) = 8 \text{ keV}, T_e(0) = 10 \text{ keV},$ $\beta = 4.1\%$ 48 min operation at 1.2 MW, 2 keV

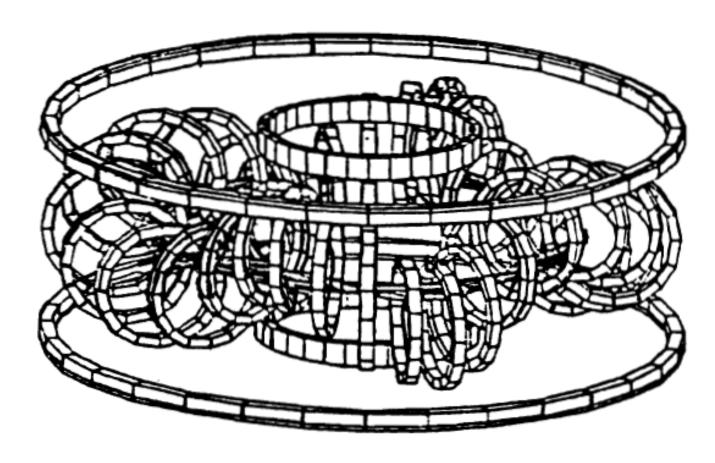
2017 upgrades of NBI, ECH, ICRF, Deuterium





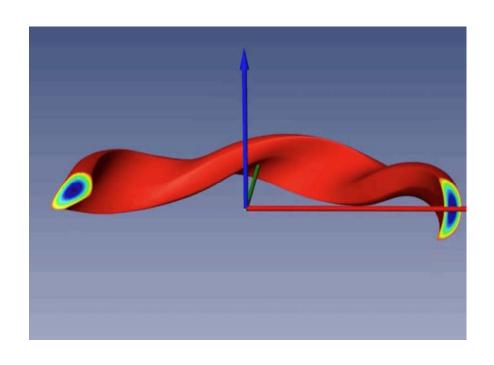
LHD @ Japan

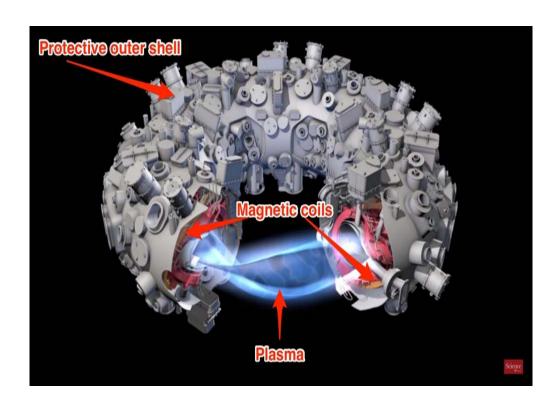
Heliac/Helias



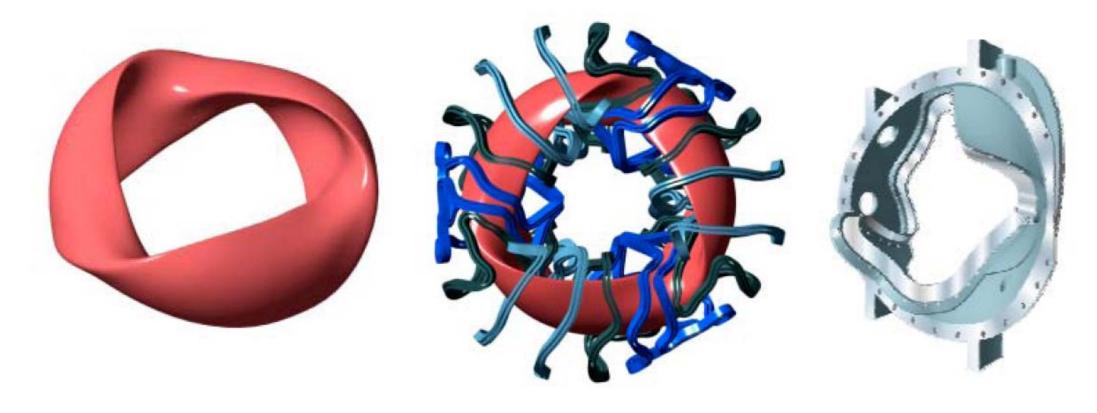
空间磁轴仿星器 Tj-II Heliac 的线圈布设

Wendelstein 7-X Modular Stellarator



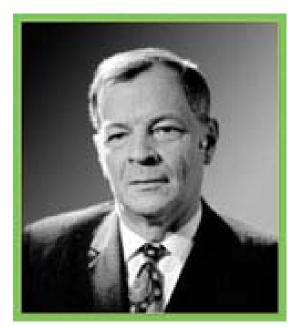


NCSX

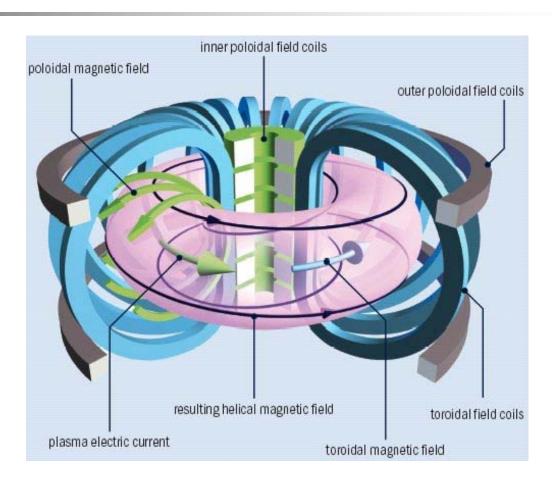


NCSX (expected in 07 but shutdown in 08)

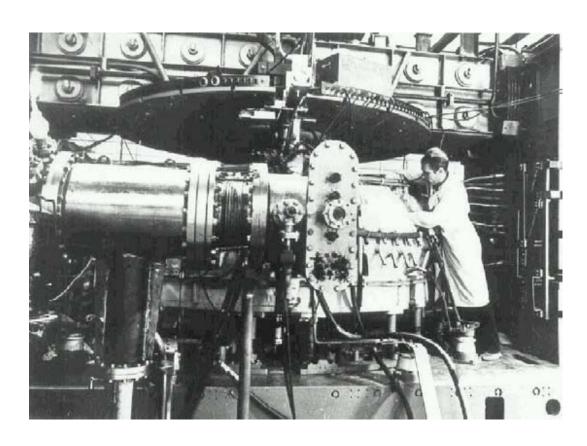
托卡马克Tokamak(领跑者)

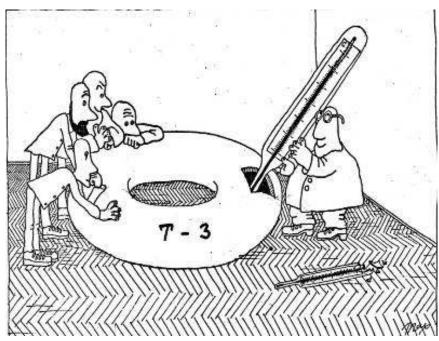


Lev Artsimovich (1909-1973)



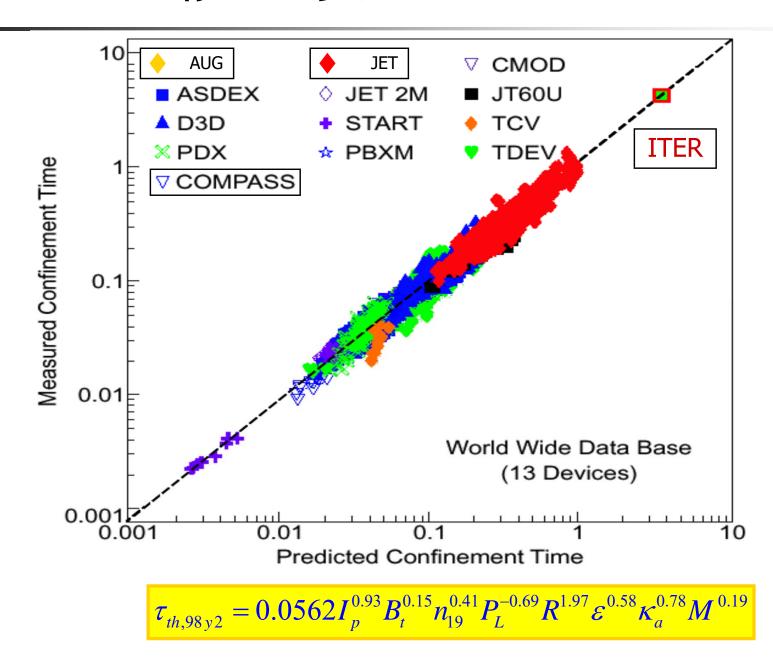
 托卡马克(TOKAMAK)在俄语中是"环形"、"真空"、"磁"、"线圈" 几个词的组合,即环流磁真空室的缩写





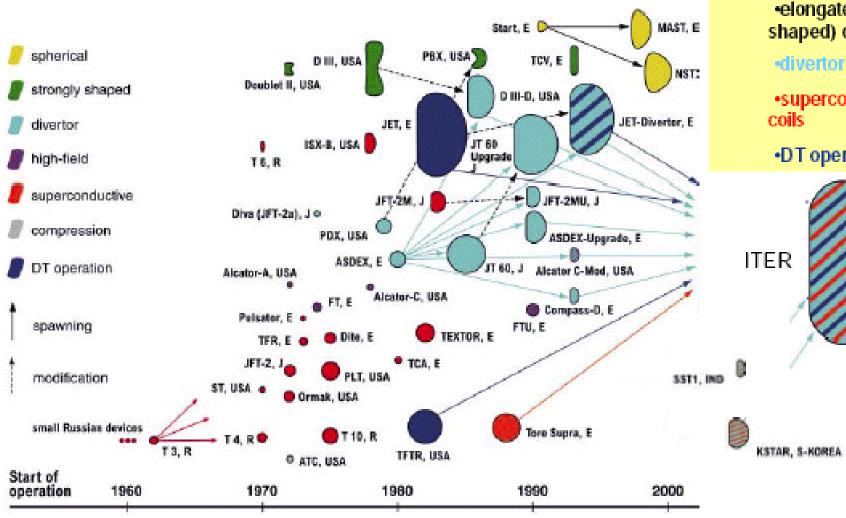
T-3 Tokamak @ USSR

Tokamak能量约束



tokamak research is mature for the step to a burning plasma - (1)

Mayor Tokamak Facilities



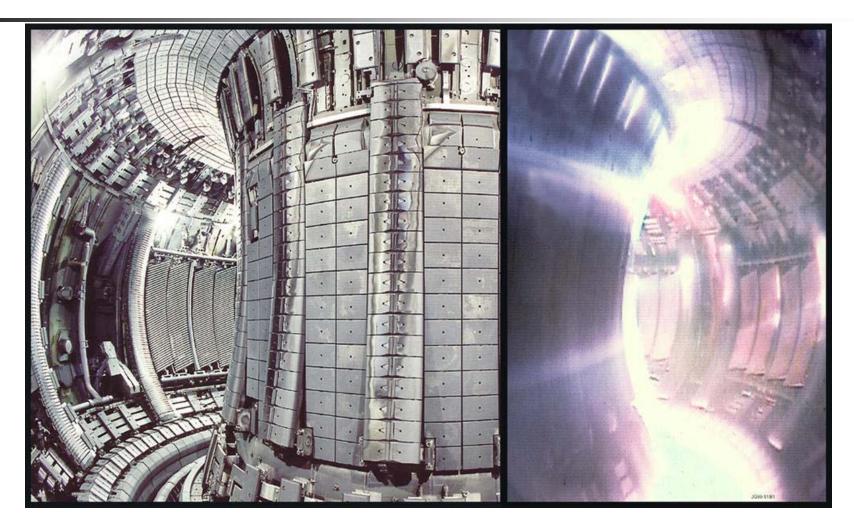
tokamak research has converged

ITER incorporates all successfull developments:

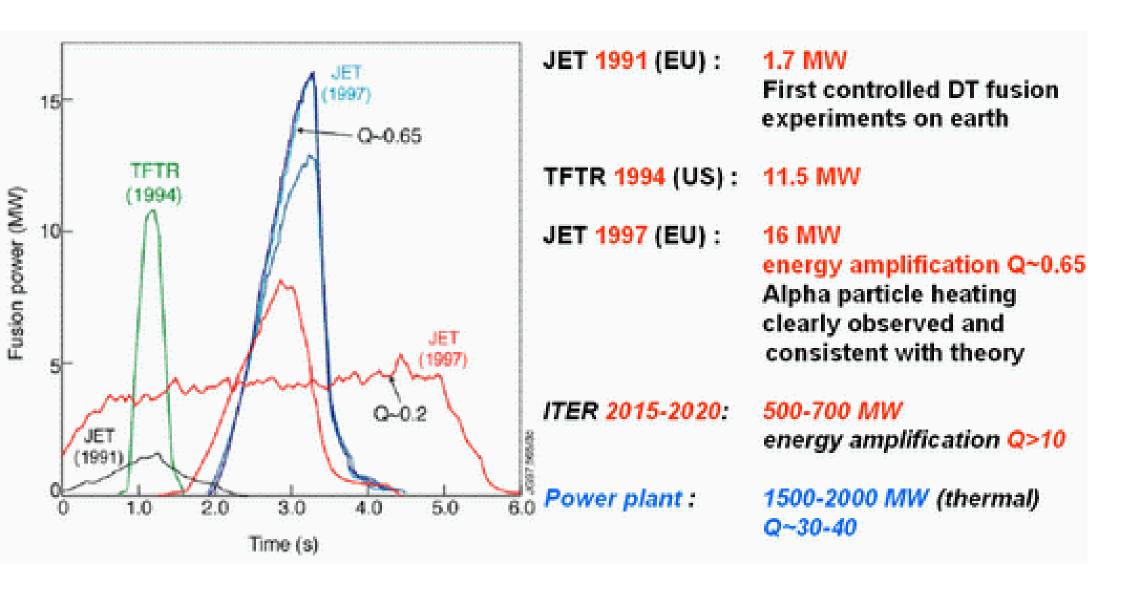
> elongated (Dshaped) cross-section

superconducting

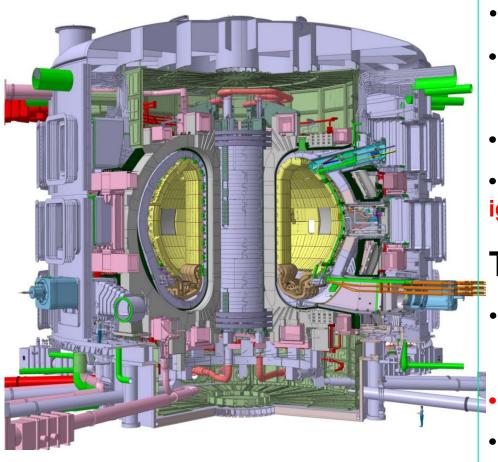
DT operation



Joint Europe Torus



International Thermonuclear Experimental Reactor



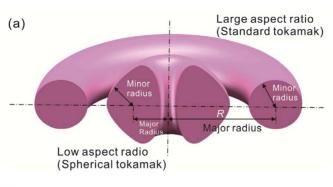
Physics:

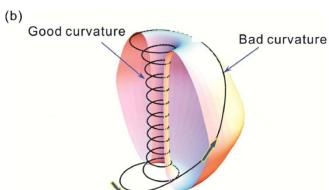
- produce a plasma dominated by α -particle heating
- produce a significant fusion power amplification factor
 (Q ≥10) in long-pulse operation (300 500s)
- aim to achieve steady-state operation of a tokamak (Q=)
- retain the possibility of exploring 'controlled ignition'(Q≥30)

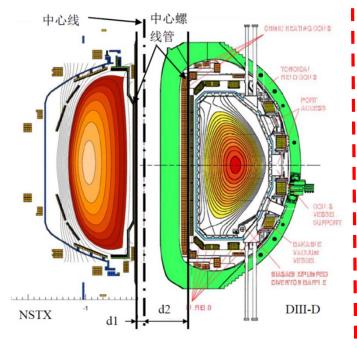
Technology:

- demonstrate integrated operation of technologies for a fusion power plant
- test components required for a fusion power plant
- test concepts for a tritium breeding module

球形托卡马克/紧凑环







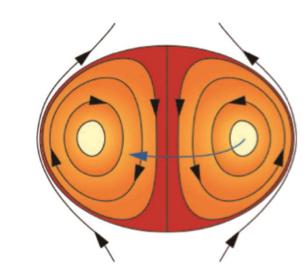


Fig. 2. Spheromak configuration. (Courtesy of Lawrence Livermore National Laboratory).



Matter and Radiation at Extremes 1 (2016) 153-162

www.journals.elsevier.com/matter-and-radiation-at-extremes

Review article

Compact magnetic confinement fusion: Spherical torus and compact torus

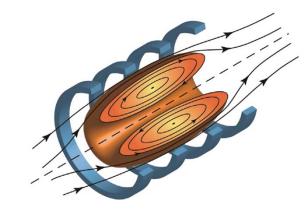
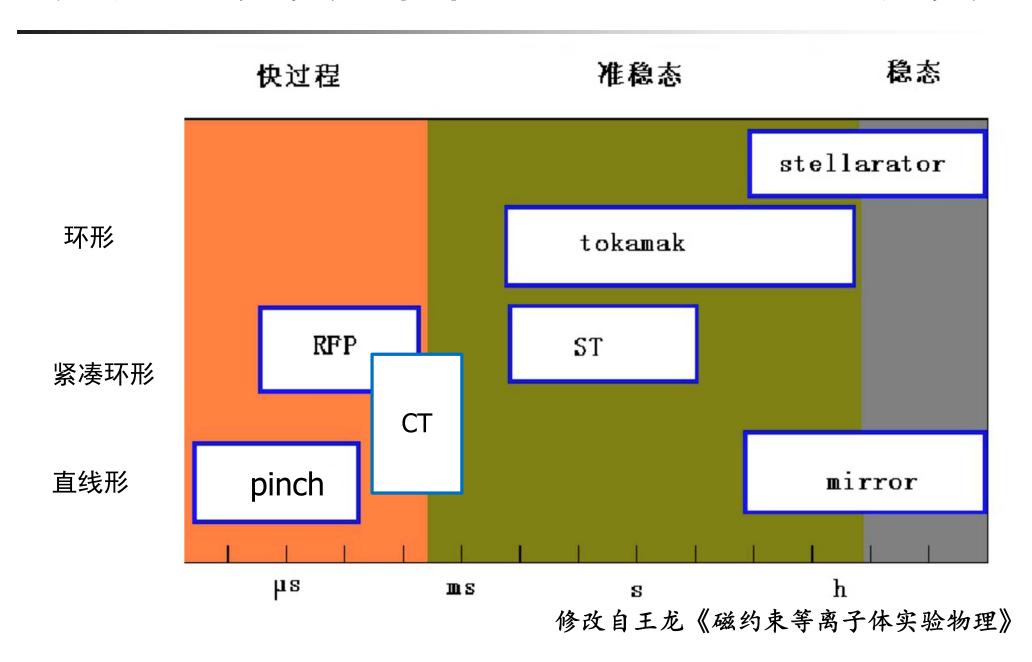
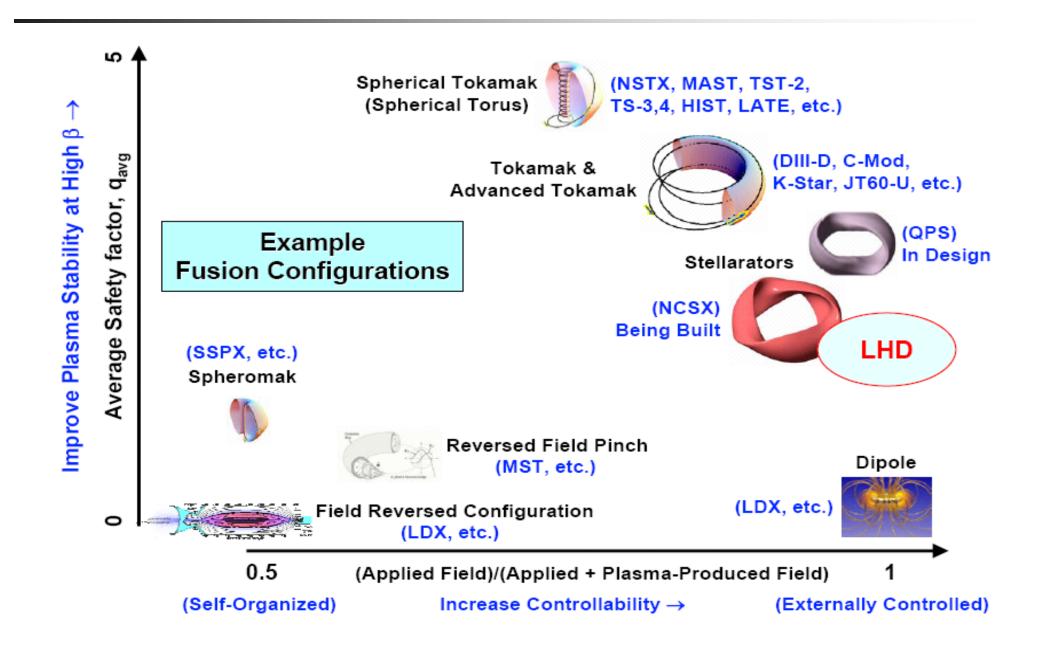


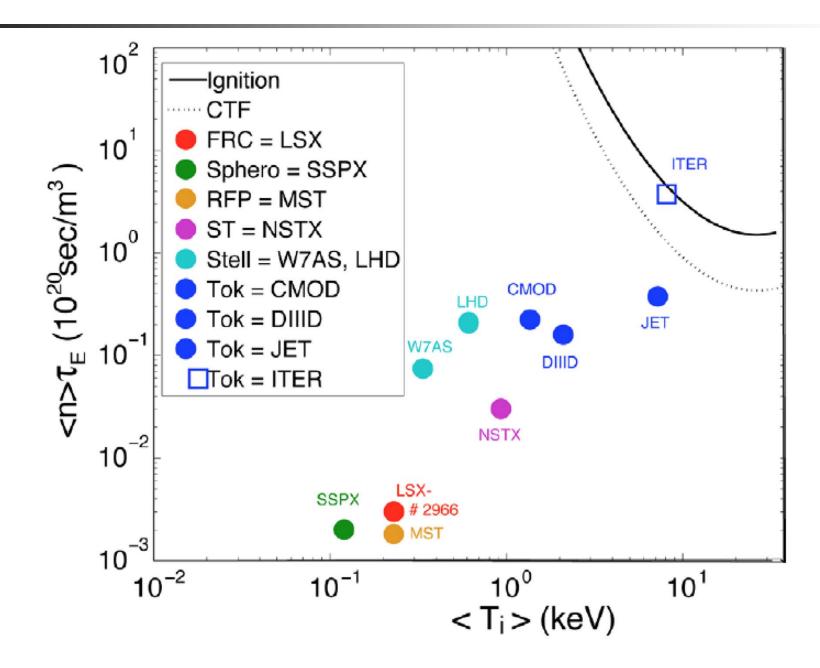
Fig. 3. Field reversed configuration (Reprint from Figs. 2-5 of Ref. [50])

磁约束聚变类型在位形-时间平面上的分类

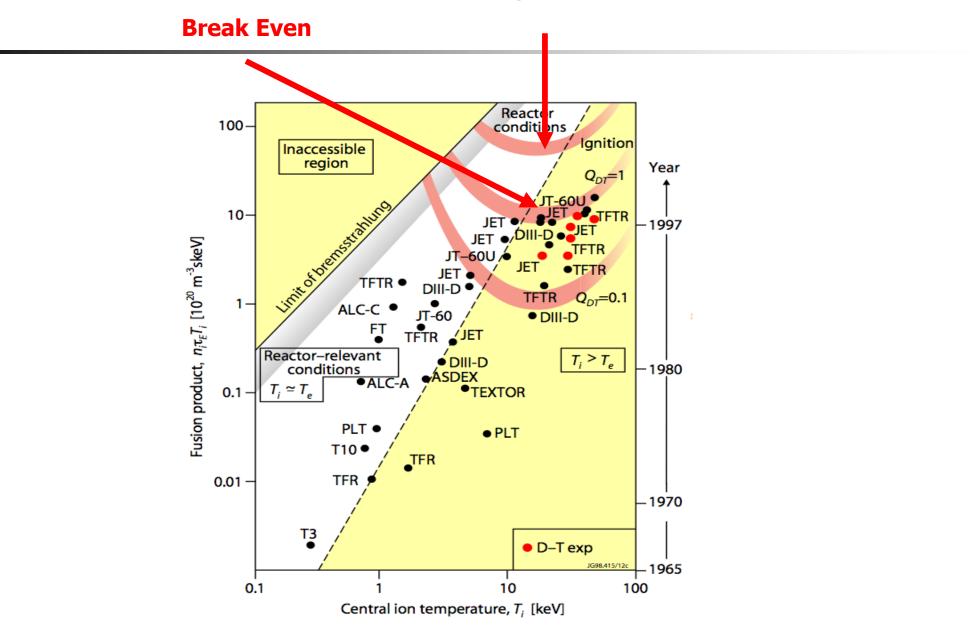


从所加磁场和等离子体稳定性所做的分类





ITER Ignition



作业 (网络学堂)

• 1. 大环径比(r/R₀<<1) 圆截面托卡马克的磁场可以表示成

$$\vec{B}(r,\theta) = \frac{rB_0}{q(r)R_0}\vec{e}_\theta + \frac{B_0}{1 + r\cos\theta/R_0}\vec{e}_\varphi$$

从直圆柱扭曲模不稳定性条件给出安全因子q应满足的条件。

- 2. 简述托卡马克和环箍缩的异同点
- 3. 选作:
 - 阅读文章【Y Xu, A general comparison between tokamak and stellarator plasmas,
 Matter and Radiation at Extremes 1,192-200 (2016)】, 比较托卡马克和仿星器的异同点,分析它们各自的优缺点。
 - 阅读文章【Z Gao, Compact magnetic confinement fusion: Spherical torus and compact torus, Matter and Radiation at Extremes 1,153-162 (2016)】, 比较球形环和 紧凑环的异同点,分析它们各自的优缺点

雨课堂期中测验

- 30分钟答题时间
- 闭卷, 雨课堂会监视切屏
- 注意单选和多选,多选题少选可得部分得分